(TG1) Multi-ion group report



Nov. 29, 2022 (G. Motojima)

Date: Nov. 25, 2022 Time: 9:51-15:30 Shot#: 184390-184497 (108shots) Prior wall conditioning: No Divertor pump: No Gas puff: D₂, H₂ H pellet: NO NBI#(1, 2, 3, 4, 5) = gas(D, D, D, D, D) = P(2.1,2.2,0.8,4.2,3.8)MWECH(77 GHz) = ant(5.5 - Uout, 2 - OUR) = P(703,0) kWECH(154 GHz) = ant(2-OLL, 2-OUL, 2-OLR) = P(723,799,986) kW ECH(56 GHz) = ant(1.5U) = P(-) kWICH(3.5U, 3.5L, 4.5U, 4.5L) = P(0,0,0,0) MWNeutron yield integrated over the experiment = 1.8×10^{17} (TG1)

Topics

- 1. Wall D/H recycling process (G. Motojima)
- 2. Isotope mass effects on sustainment of e-ITB plasma (N. Kenmochi)
- 3. Exposure of W samples to divertor plasma (C.P. Dhard, D. Naujoks (IPP), S. Masuzaki)

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Wall D/H recycling process (G. Motojima)



✓ R_{ax} =3.6m, B=2.75T, γ=1.254, Bq=100%

✓ Working gas: H_2 , D_2 , SSGP(H_2)

Motivation

To understand the wall recycling process, the plan was to inject H-pellets after the D-wall, but due to a problem of the pellet injector, SSGPs were used.

✓ Results

- The D/H+D measured by passive spectroscopy is 1 showing D-rich. In addition, all the injection gases in the NBI are Deuterium.
- An example of a discharge before SSGP(H) injection is shown. It is curious to see that, although n_D is almost dominant in the early stages of the discharge, in the NBI phase, n_D has a hollow distribution and n_H has a central flat distribution. This means that in the discharge before the SSGP is injected, there is already hydrogen recycling from the wall and hydrogen is supplied to the plasma core.
- Also, the reason for the non-mixing state has not yet been explained and will be analysed in detail including SSGP(H) shots.

G. Motojima

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Isotope mass effects on sustainment of e-ITB plasma



Exposure of W samples to divertor plasma

Shot #: 184470 - 184497

 $(R_{ax}, B_t, \gamma, B_q) = (3.6 \text{ m}, 2.75 \text{ T}, 1.2538, 100.0\%)$

Working gas: D2

 $\mathsf{P}_{\mathsf{NBI-1}} \thicksim 2.0$ MW, $\mathsf{P}_{\mathsf{NBI-2}} \thicksim 2.8$ MW, $\mathsf{P}_{\mathsf{NBI-3}} \thicksim 2.2$ MW

Motivation

- Tungsten is appearing as a potential material for fusion reactor applications. However, because of its hardness and brittleness, it is not so easy to manufacture. W-alloys are being explored to overcome these problems.
- In this experiment, W-alloys samples are exposed to the LHD divertor plasma to investigate the effects of the deuterium plasma exposure on samples properties.

Results

- Three sets of W alloy samples (W95%-Cu/Ni, W95%-Fe/Ni, W97%-Fe/Ni) and a set of pure W were exposed to divertor plasma using the manipulator at a 10.5L port.
- Total ~ 140s exposure was conducted.
- Line averaged density was kept to be \sim 2E19/m3.
- Surface analyses will be conducted as soon as possible.



C.P. Dhard, D. Naujoks (IPP), S. Masuzaki